

# Mathematics of Radiation Transport Modelling

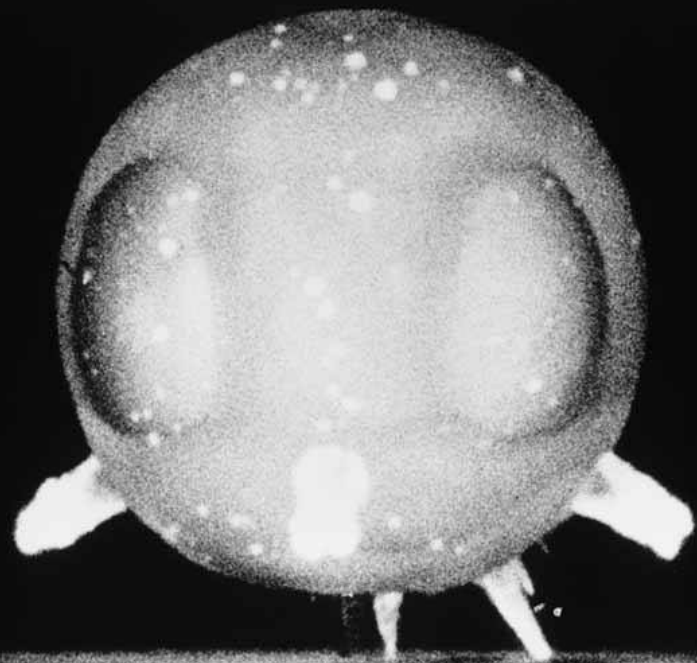
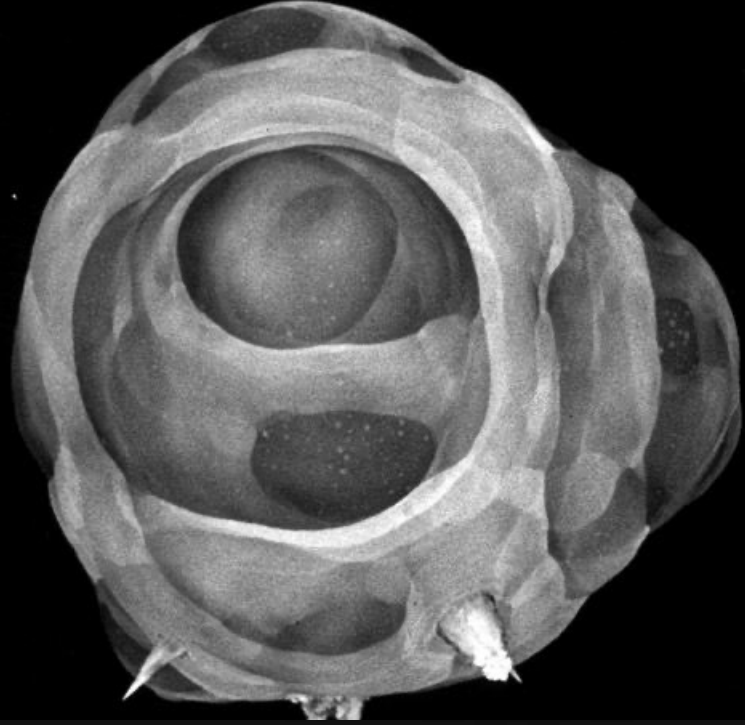
through the eyes of a probabilist

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<https://mathrad.ac.uk/>



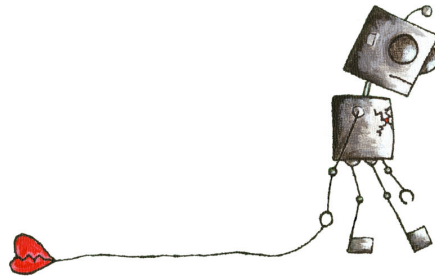
# Radiation transport equations

## Boltzmann transport equation

Let  $\psi = \psi(\mathbf{y}) = \psi(t, \mathbf{x}, \boldsymbol{\Omega}, e) : \mathbb{R}^7 \rightarrow \mathbb{R}$  denote angular flux

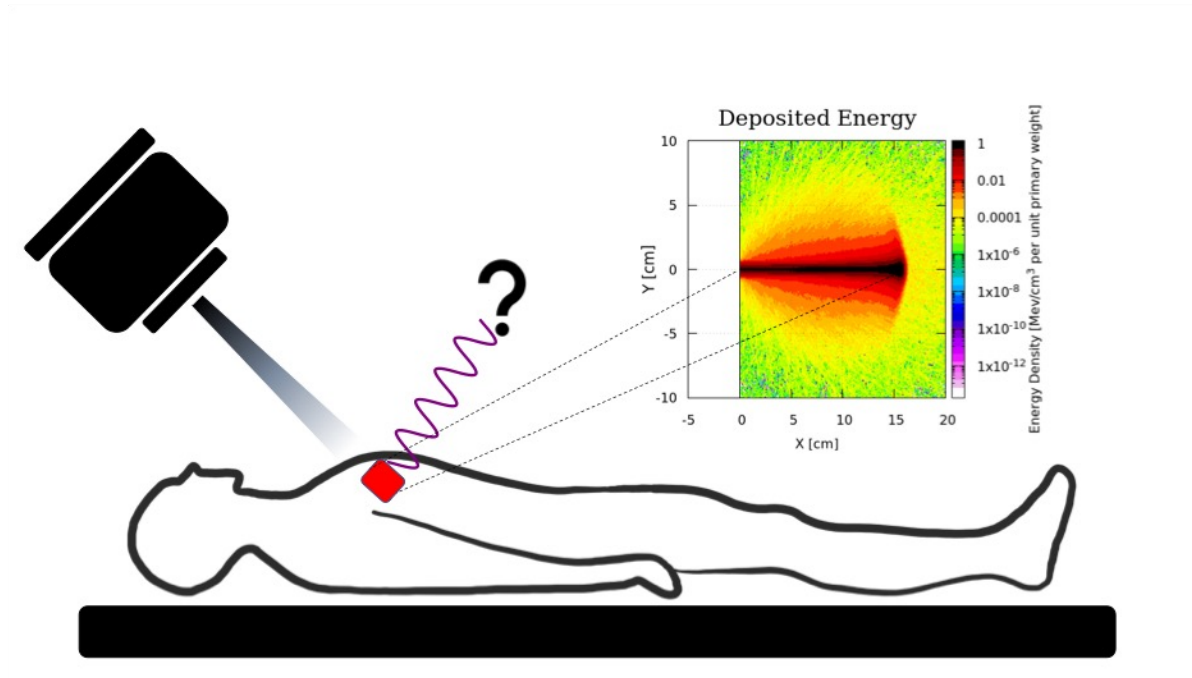
$$\underbrace{\partial_t \psi(\mathbf{y}) + \boldsymbol{\Omega} \cdot \nabla_{\mathbf{x}} \psi(\mathbf{y})}_{\text{Transport}} + \underbrace{\sigma_T(\mathbf{x}, e)}_{\text{Total cross section}} \psi(\mathbf{y}) = \int_{e'} \int_{\boldsymbol{\Omega}'} \underbrace{\sigma_S(\mathbf{x}, \boldsymbol{\Omega}' \rightarrow \boldsymbol{\Omega}, e' \rightarrow e)}_{\text{Scattering cross section}} \psi(t, \mathbf{x}, \boldsymbol{\Omega}', e') d\boldsymbol{\Omega}' de'$$

+ BCs, ICs, source terms



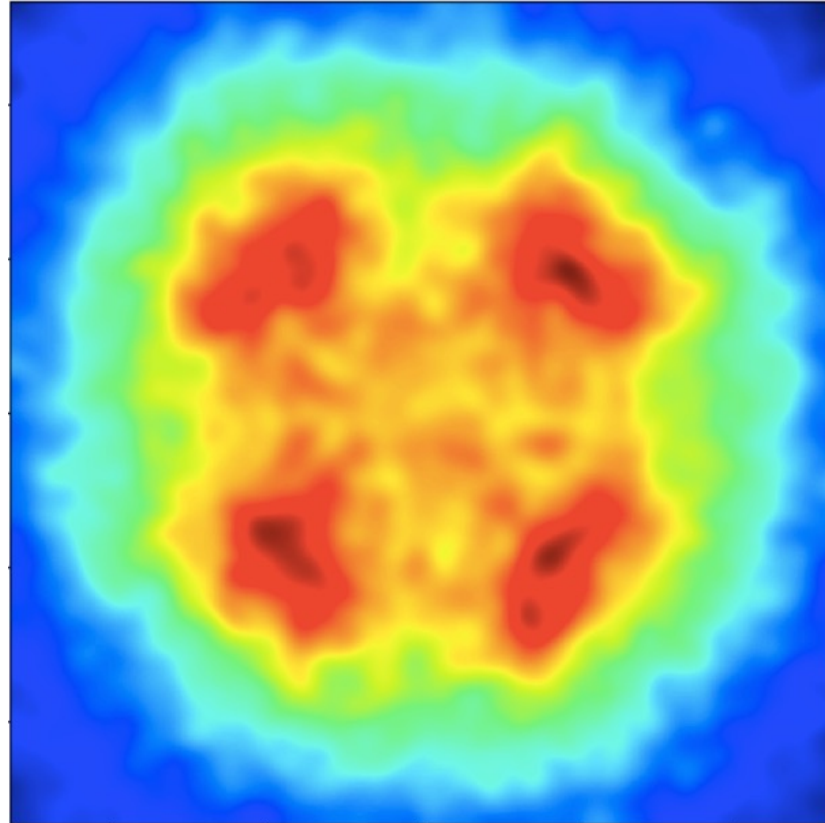
# Types of radiation transport problems

- Proton beam forward modelling

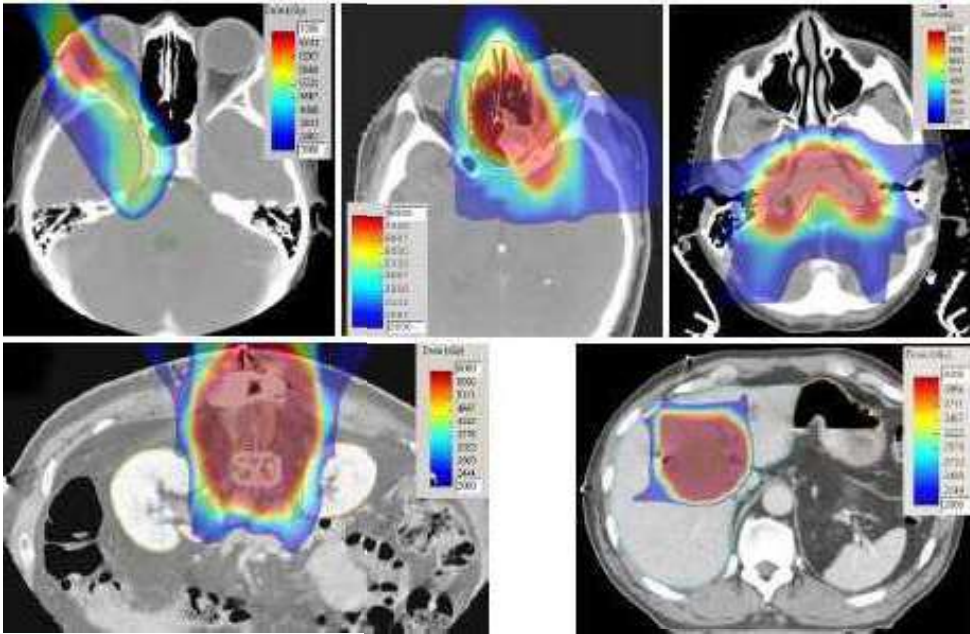


# Types of radiation transport problems

## ➤ Criticality



# Benefits of PBT

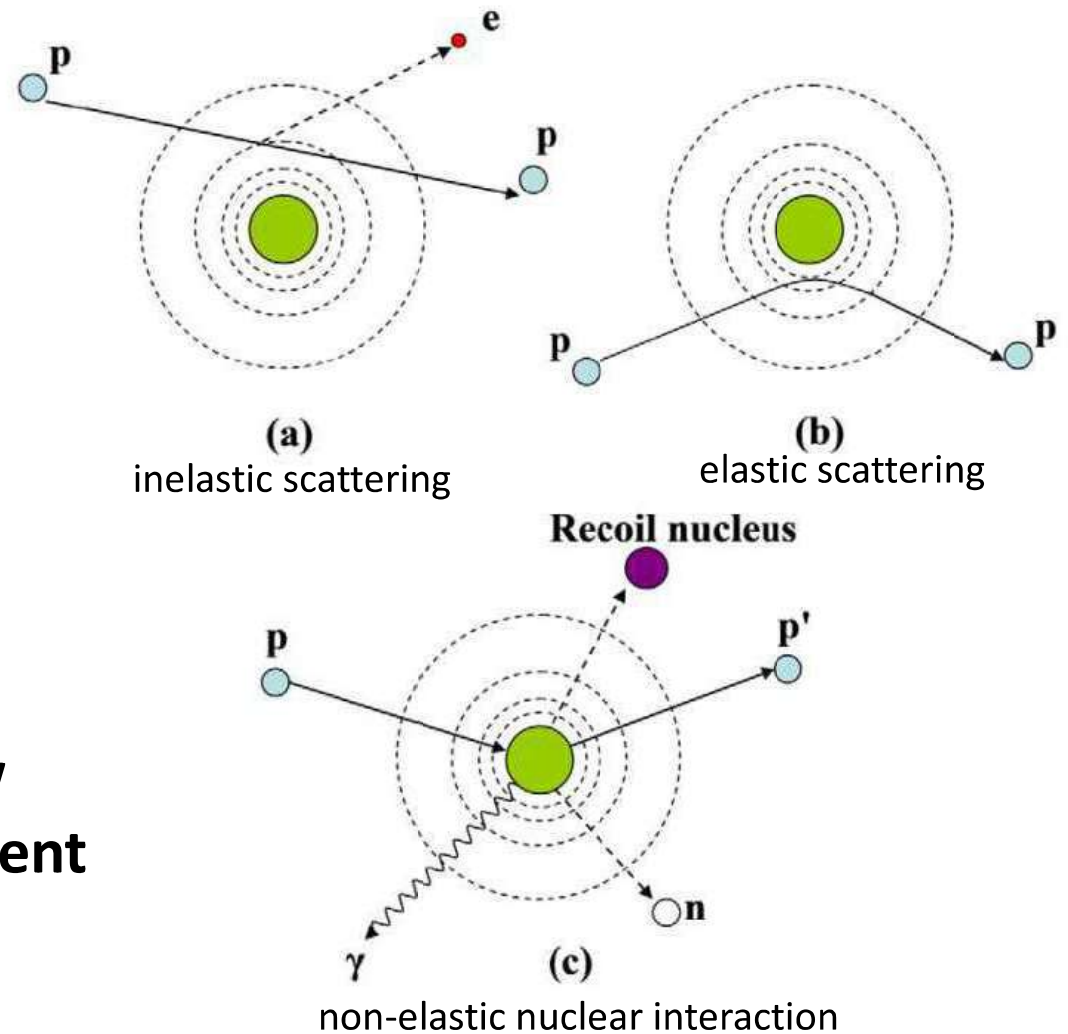


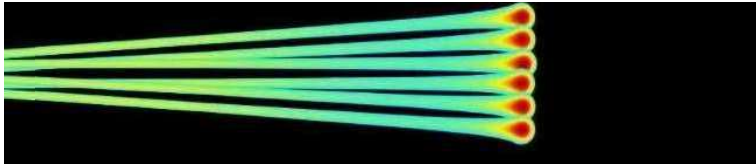
- Spare healthy tissue- reduce risk of secondary malignancies
- Escalate the dose to the target to curative levels
- Re-irradiation settings

# Interactions

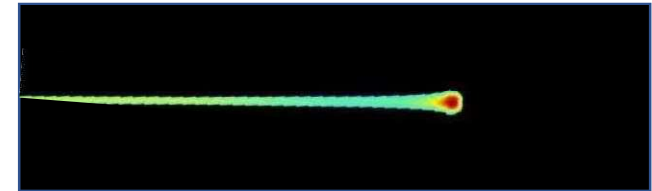
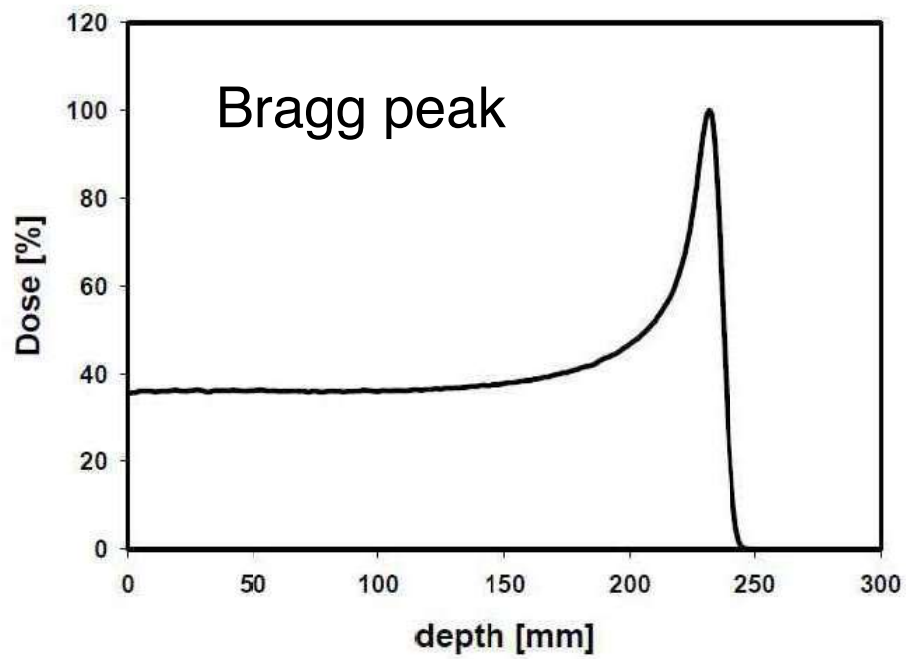
- Ionization (Coulomb effect)
- Coulomb interactions with atomic nucleus
- Nuclear interactions with atomic nucleus

**These interactions govern how protons deposit their dose in patient fundamentally**





Slow loss of energy due to  
Coulomb interactions with  
atomic electrons



$$\frac{dE}{dx} \propto \frac{1}{v^2} \left( \frac{Z}{A} \right) z^2$$

Bethe-Block

## PROTON BEAM SDE

A special kind of Stochastic Differential Equation models the energy deposition of individual **proton streams**:  $Y_\ell = (\epsilon_\ell, r_\ell, \Omega_\ell)$

- ▶  $\epsilon_\ell$  is the energy of the proton stream after it has traversed a distance  $\ell$
- ▶  $r_\ell$  is the position of the proton stream after it covers a distance  $\ell$
- ▶  $\Omega_\ell$  is the direction of travel of the proton after it covers a distance  $\ell$ .

$$\epsilon_\ell = \epsilon_0 - \int_0^\ell \varsigma(Y_{l-}) dl - \int_0^\ell (1-u)\epsilon_{l-} N_{ne}(Y_{l-}; dl, d\Omega', du)$$

$$r_\ell = r_0 + \int_0^\ell \Omega_l dl$$

$$\begin{aligned} \Omega_\ell &= \Omega_0 - \int_0^\ell m(Y_l)^2 \Omega_l dl + \int_0^\ell m(Y_{l-}) \Omega_l \wedge dB_l \\ &\quad + \int_0^\ell \int_{\mathbb{S}_2} (\Omega' - \Omega_{l-}) N_e(Y_{l-}; dl, d\Omega') + \int_0^\ell \int_0^1 \int_{\mathbb{S}_2} (\Omega' - \Omega_{l-}) N_{ne}(Y_{l-}; dl, d\Omega', du) \end{aligned}$$

## WHERE'S THE MATH?

- ▶ Does does occupation measure of the solution  $(\epsilon_\ell, r_\ell, \Omega_\ell)$  to this SDE have a density  $r(z)$  with respect to Lebesgue measure on  $\Upsilon = (0, \infty) \times D \times \mathbb{S}_2$ ?
- ▶ **Important because:** We can define for a test function  $f$  on  $\Upsilon = (0, \infty) \times D \times \mathbb{S}_2$  (the configuration space of the solution), the 'interrogation' potential of where (and how much) energy is deposited along its stochastic path:

$$U[f] = -\mathbb{E} \left[ \int_0^\Lambda f(Y_{\ell-}) d\epsilon_\ell \right] = - \int_\Upsilon f(z) r(z) \left\langle \frac{dE}{dx} \right\rangle dz,$$

where  $r(z)$  is the occupation density of our SDE.

here  $\Lambda$  is the total distance covered by the proton stream and  $Y_\ell = (\epsilon_\ell, r_\ell, \Omega_\ell)$

- ▶ If we define

$$\begin{aligned} D[f] &:= - \lim_{\epsilon \rightarrow 0} \frac{1}{\epsilon} \mathbb{E} \left[ \int_0^\Lambda \left( f(r_\ell + \epsilon \Omega_\ell) - f(r_\ell) \right) d\epsilon_\ell \right] \\ &= \int_\Upsilon \Omega \cdot \nabla_r f(r) u(z) dz. \end{aligned}$$

where  $u(z) = r(z) \left\langle \frac{dE}{dx} \right\rangle$

- ▶ Because of the existence of the density, we can appeal to duality to tell us that

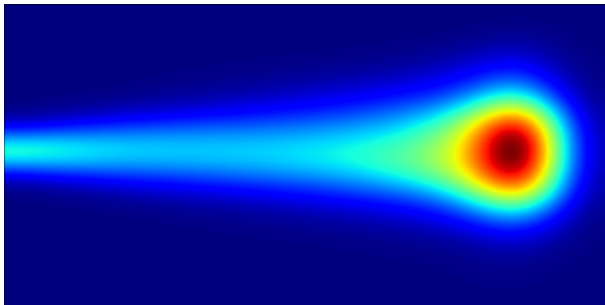
$$D[f] = \langle (\Omega \cdot \nabla_r) f, u \rangle = - \langle f, (\Omega \cdot \nabla_r) u \rangle.$$

## BRAGG MANIFOLD

We defined the *path Bragg manifold* to be the quantity

$$b(z) = -\Omega \cdot \nabla_r u(z).$$

As alluded to above, this is the average rate of directional energy deposition at configuration  $z = (\epsilon, r, \Omega) \in \Upsilon$  in the sequential proton track.



## WHERE'S THE MATH?

- ▶ Does does occupation measure of the solution  $(\epsilon_\ell, r_\ell, \Omega_\ell)$  to this SDE have a density  $r(z)$  with respect to Lebesgue measure on  $\Upsilon = (0, \infty) \times D \times \mathbb{S}_2$ ?
- ▶ Strip away the jumps from the SDE

$$\epsilon_\ell = \epsilon_0 - \int_0^\ell \varsigma(Y_{l-})dl - \int_0^\ell (1-u)\epsilon_{l-}N_{ne}(Y_{l-}; dl, d\Omega', du)$$

$$r_\ell = r_0 + \int_0^\ell \Omega_l dl$$

$$\begin{aligned} \Omega_\ell &= \Omega_0 - \int_0^\ell m(Y_l)^2 \Omega_l dl + \int_0^\ell m(Y_{l-}) \Omega_l \wedge dB_l \\ &\quad + \int_0^\ell \int_{\mathbb{S}_2} (\Omega' - \Omega_{l-}) N_e(Y_{l-}; dl, d\Omega') + \int_0^\ell \int_0^1 \int_{\mathbb{S}_2} (\Omega' - \Omega_{l-}) N_{ne}(Y_{l-}; dl, d\Omega', du) \end{aligned}$$

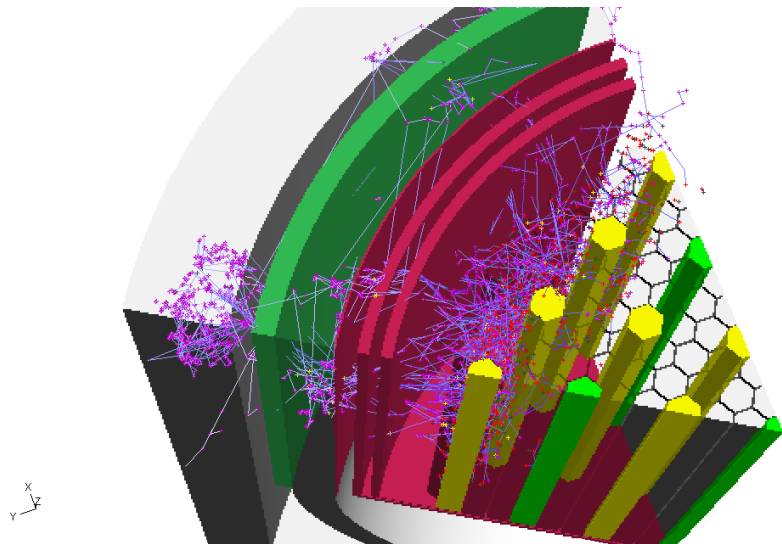
- ▶ Write the SDE in the following form

$$dY_\varepsilon = \sum_{j=1}^3 X_j(t, Y_\varepsilon) dW_\varepsilon^j + X_0(t, Y_\varepsilon) dt, \quad t \geq 0,$$

- ▶ Hörmander's theorem tells us that solution to reduced SDE has a transition density if

$$\dim \text{span Lie}\{\partial_\epsilon + X_0, X_1, X_2\} = 6,$$

# Nuclear reactor core modelling



## NEUTRON TRANSPORT EQUATION

Neutron flux is thus identified as  $\Psi_g : D \times V \rightarrow [0, \infty)$ , which solves the integro-differential equation

$$\begin{aligned} & \frac{\partial \Psi_g}{\partial t}(t, r, v) + v \cdot \nabla \Psi_g(t, r, v) + \sigma(r, v) \Psi_g(t, r, v) \\ &= \int_V \Psi_g(r, v', t) \sigma_s(r, v') \pi_s(r, v', v) dv' + \int_V \Psi_g(r, v', t) \sigma_f(r, v') \pi_f(r, v', v) dv', \end{aligned}$$

where the different components are measurable in their dependency on  $(r, v)$  and are explained as follows:

$\sigma_s(r, v')$  : the rate at which scattering occurs from incoming velocity  $v'$ ,

$\sigma_f(r, v')$  : the rate at which fission occurs from incoming velocity  $v'$ ,

$\sigma(r, v)$  : the sum of the rates  $\sigma_f + \sigma_s$  and is known as the cross section,

$\pi_s(r, v', v) dv'$  : the scattering yield at velocity  $v$  from incoming velocity  $v'$ ,  
satisfying  $\pi_s(r, v, V) = 1$ ,

$\pi_f(r, v', v) dv'$  : the average neutron yield at velocity  $v$  from fission with  
incoming velocity  $v'$ , satisfying  $\pi_f(r, v, V) < \infty$

We will assume that all quantities are uniformly bounded away from zero and infinity.

## BOUNDARY CONDITIONS

- ▶ Boundary conditions which represent 'fission containment'

$$\begin{cases} \Psi_g(0, r, v) = g(r, v) & \text{for } r \in D, v \in V, \text{ (initial condition)} \\ \Psi_g(t, r, v) = g(r, v) = 0 & \text{for } r \in \partial D \text{ if } v \cdot \mathbf{n}_r < 0, \text{ (neutron annihilation)} \end{cases}$$

- ▶  $\mathbf{n}_r$  is the outward facing normal of  $D$  at  $r \in \partial D$
- ▶  $g : D \times V \rightarrow [0, \infty)$  is a bounded, measurable function which we will later assume has some additional properties.

## (FORWARD $\rightarrow$ BACKWARDS) NEUTRON TRANSPORT EQUATION

- ▶ Hence, with similar computations, this tells us that, for  $f, g \in L^2(D \times V)$ ,

$$\langle f, (\mathcal{T} + \mathcal{S} + \mathcal{F})g \rangle = \langle (\mathcal{T} + \mathcal{S} + \mathcal{F})f, g \rangle,$$

where

$$\left\{ \begin{array}{ll} \mathcal{T}f(r, v) & := v \cdot \nabla f(r, v) & \text{(backwards transport)} \\ \mathcal{S}f(r, v) & := \sigma_s(r, v) \int_V f(r, v') \pi_s(r, v, v') dv' - \sigma_s(r, v) f(r, v) & \text{(backwards scattering)} \\ \mathcal{F}f(r, v) & := \sigma_f(r, v) \int_V f(r, v') \pi_f(r, v, v') dv' - \sigma_f(r, v) f(r, v) & \text{(backwards fission)} \end{array} \right.$$

- ▶ This leads us to the so called *backwards neutron transport equation* (which is also known as the *adjoint neutron transport equation*) given by the Abstract Cauchy Problem on  $L^2(D \times V)$ ,

$$\frac{\partial \psi_g}{\partial t}(t, \cdot, \cdot) = (\mathcal{T} + \mathcal{S} + \mathcal{F})\psi_g(t, \cdot, \cdot)$$

with additional boundary conditions

$$\left\{ \begin{array}{ll} \psi_g(0, r, v) = g(r, v) & \text{for } r \in D, v \in V, \\ \psi_g(t, r, v) = 0 & \text{for } r \in \partial D \text{ if } v \cdot \mathbf{n}_r > 0. \end{array} \right.$$

## UNDERLYING STOCHASTICS (LEADING TO MONTE-CARLO)

- ▶ Backwards equation lends itself well to stochastic representation,

$$\begin{aligned} \frac{\partial \psi_g}{\partial t}(t, r, v) &= v \cdot \nabla \psi_g(t, r, v) - \sigma(r, v) \psi_g(t, r, v) \\ &\quad + \sigma_s(r, v) \int_V \psi_g(r, v', t) \pi_s(r, v, v') dv' + \sigma_f(r, v) \int_V \psi_g(r, v', t) \pi_f(r, v, v') dv'. \end{aligned}$$

- ▶ The physical process of fission is a Markov-additive branching process (*neutron branching process*).
- ▶ Represented by a configuration of physical location and velocity of particles in  $D \times V$ , say  $\{(r_i(t), v_i(t)) : i = 1, \dots, N_t\}$ , where  $N_t$  is the number of particles alive at time  $t \geq 0$ .
- ▶ Represent as a process in the space of the atomic measures

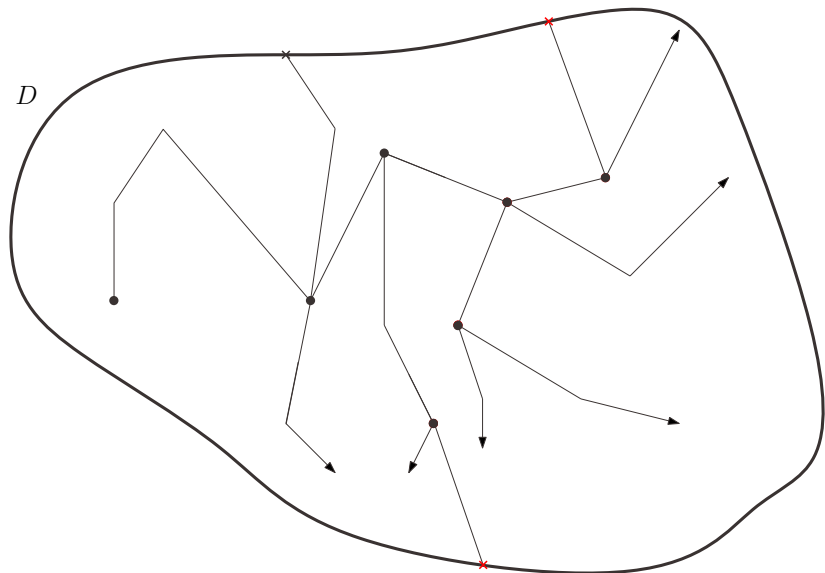
$$X_t(A) = \sum_{i=1}^{N_t} \delta_{(r_i(t), v_i(t))}(A), \quad A \in \mathcal{B}(D \times V), \quad t \geq 0,$$

where  $\delta$  is the Dirac measure, define on  $\mathcal{B}(D \times V)$ , the Borel subsets of  $D$ .

- ▶ Then the stochastic representation of the backwards NTE is nothing more than

$$\phi_t[g](r, v) = \mathbb{E}_{\delta_{(r, v)}}[g, X_t] = \mathbb{E}_{\delta_{(r, v)}} \left[ \sum_{i=1}^{N_t} g(r_i(t), v_i(t)) \right], \quad t \geq 0.$$

## NEUTRON BRANCHING PROCESS



## $\lambda$ -EIGENVALUE PROBLEM

- ▶ So far

$$\langle f, \phi_t[g] \rangle = \langle \Psi_f(t, \cdot, \cdot), g \rangle$$

for all  $f, g \in L_2(D \times V)$

- ▶ We want to play with the eigenfunction  $\tilde{\varphi} \in L_2(D \times V)$ , e.g.

$$\langle f, \phi_t[\tilde{\varphi}] \rangle = \langle \Psi_f(t, \cdot, \cdot), \tilde{\varphi} \rangle = e^{\lambda t} \langle f, \tilde{\varphi} \rangle$$

suggesting (at least in the  $L_2(D \times V)$  sense)

$$\phi_t[\tilde{\varphi}](r, v) = \mathbb{E}_{\delta_{(r,v)}}[\langle \tilde{\varphi}, X_t \rangle] := e^{\lambda t} \tilde{\varphi}(r, v)$$

⇒ points us towards Monte-Carlo methods - especially when  $\lambda = 0$

# PERRON-FROBENIUS

## Theorem (Horton, K., Villemonais, 2018)

Suppose that

- ▶  $D$  is non-empty and convex;
- ▶ Cross-sections  $\sigma_s, \sigma_f, \pi_s$  and  $\pi_f$  are uniformly bounded away from infinity;
- ▶  $\inf_{r \in D, v, v' \in V} (\sigma_s(r, v)\pi_s(r, v, v') + \sigma_f(r, v)\pi_f(r, v, v')) > 0$

Then, for the semigroup  $(\phi_t, t \geq 0)$ , there exists a  $\lambda_* \in \mathbb{R}$ , a positive<sup>1</sup> right eigenfunction  $\varphi \in L_\infty^+(D \times V)$  and a left eigenmeasure which is absolutely continuous with respect to Lebesgue measure on  $D \times V$  with density  $\tilde{\varphi} \in L_\infty^+(D \times V)$ , both having associated eigenvalue  $e^{\lambda_* t}$ , and such that  $\varphi$  (resp.  $\tilde{\varphi}$ ) is uniformly (resp. a.e. uniformly) bounded away from zero on each compactly embedded subset of  $D \times V$ . In particular, for all  $g \in L_\infty^+(D \times V)$ ,

$$\langle \tilde{\varphi}, \phi_t[g] \rangle = e^{\lambda_* t} \langle \tilde{\varphi}, g \rangle \quad (\text{resp. } \phi_t[\varphi] = e^{\lambda_* t} \varphi) \quad t \geq 0.$$

Moreover, there exists  $\varepsilon > 0$  such that

$$\sup_{g \in L_\infty^+(D \times V): \|g\|_\infty \leq 1} \left\| e^{-\lambda_* t} \varphi^{-1} \phi_t[g] - \langle \tilde{\varphi}, g \rangle \right\|_\infty = O(e^{-\varepsilon t}) \text{ as } t \rightarrow \infty.$$

<sup>1</sup>To be precise, by a positive eigenfunction, we mean a mapping from  $D \times V \rightarrow (0, \infty)$ . This does not prevent it being valued zero on  $\partial D$ , as  $D$  is an open bounded, convex domain.

## $\lambda$ -EIGENVALUE AND MONTE-CARLO LOGIC

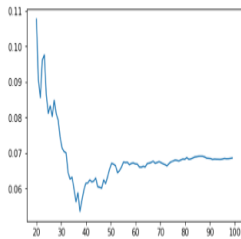
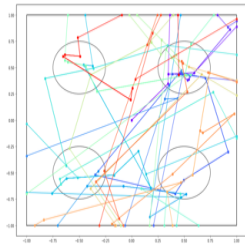
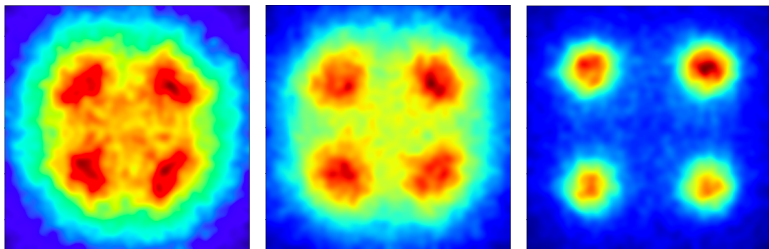
- ▶ Suppose now we can efficiently simulate the Neutron branching process, recalling that

$$\phi_t[g](r, v) := \mathbb{E}_{\delta_{(r, v)}}[\langle g, X_t \rangle], \quad t \geq 0, r \in D, v \in V,$$



$$\lambda_* = \lim_{t \rightarrow \infty} \frac{1}{t} \log \phi_t[g](r, v) = \lim_{t \rightarrow \infty} \frac{1}{t} \log \mathbb{E}_{\delta_{(r, v)}}[\langle g, X_t \rangle], \quad t \geq 0, r \in D, v \in V.$$

# Monte-Carlo, Importance Map $\tilde{\varphi}$



## MANY-TO-ONE

- ▶ The representation  $\mathcal{T} + \mathcal{S} + \mathcal{F} = \mathcal{L} + \beta$ , where

$$\mathcal{L}f(r, v) = v \cdot \nabla f(r, v, t) + \alpha(r, v) \int_V (f(r, v', t) - f(r, v, t)) \pi(r, v, v') dv'.$$

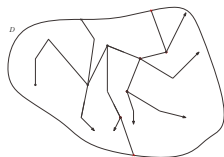
This is the Markov generator of a **neutron random walk (NRW)**  $(R, \Upsilon)$  (scatters at rate  $\alpha$  and chooses new velocity with distribution  $\pi$ ) with probabilities  $(\mathbf{P}_{(r,v)}, r \in D, v \in V)$ . We have a new representation in terms of  $(R, \Upsilon)$ ,

$$\phi_t[g](r, v) = \mathbb{E}_{\delta_{(r,v)}}[\langle g, X_t \rangle] = \mathbf{E}_{(r,v)} \left[ e^{\int_0^t \beta(R_u, \Upsilon_u) du} g(R_t, \Upsilon_t) \mathbf{1}_{(t < \tau^D)} \right],$$

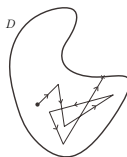
for  $t \geq 0, r \in D, v \in V$ , where

$$\tau^D = \inf\{t > 0 : R_t \notin D\}.$$

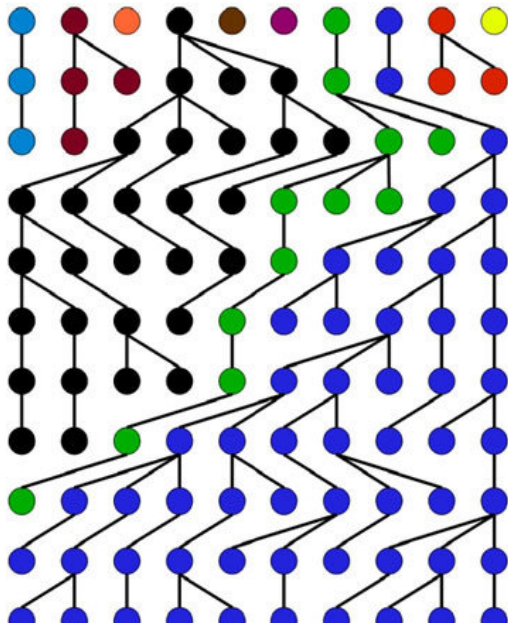
- ▶ This affords the opportunity to avoid simulating entire trees:



can be replaced by



## INTERACTING PARTICLE MONTE-CARLO



Thank you!

# PROTON BEAM FACILITY UCLH

